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41 42	1000.APPEN	DIX A Concentrations in Air Above Natural Background
43	AUTHORITY	Y: Implementing Section 25b and authorized by Section 27 of the Environmental

Protection	Act [415 ILCS 5/25b and 27].
2(B) at 10	Adopted in R82-2 at 9 Ill. Reg. 19391, effective December 4, 1985; amended in R82-Ill. Reg. 12938, effective July 21, 1986; amended in R18-28 at 47 Ill. Reg,
	SUBPART A: GENERAL PROVISIONS
Section 10	000.101 Authority
	ion Control Board adopts the rules and regulations contained in this title
_	Rev. Stat. 1983, ch. 111½ 2, par. 1025(b)).
(Sc	ource: Amended at 47 Ill. Reg, effective)
Section 10	000.102 Purpose and Policy
a)	This The regulations in this Part establishes establish standards for protection
,	against radiological air pollutants associated with materials and activities under
	licenses issued by the United States Nuclear Regulatory Commission (NRC)
	underpursuant to the Atomic Energy Act of 1954 (42 U.S.C. 5801 et seq.) as
	amended, and the Energy Reorganization Act of 1974 (42 U.S.C. 5801 et seq.)
b)	In addition to complying with the other applicable requirements of this Part,
,	persons subject to this Part must It is the policy of the Pollution Control Board that
	persons subject to this Part shall, in addition to complying with the requirements
	of this Part, make every reasonable effort to maintain radiation exposures in, and
	releases of radioactive materials to, unrestricted areas as low as is reasonably
	achievable. The term "as low as is reasonably achievable" means as low as is
	reasonably achievable <u>considering</u> taking into account the state of technology, the economics of improvements in relation to benefits to the public health and safety,
	and other societal and socioeconomic considerations, in relation to the utilization
	of atomic energy in the public interest.
<u>c)</u>	Persons licensed by the NRC United States Nuclear Regulatory Commission to
	operate light-water-cooled nuclear power reactors willshall be deemed to satisfy
	the requirements of this subsection (b) if they achieve the design objectives and
	limiting conditions for operation specified set out in 10 CFR 50, Appendix I (1984), incorporated by reference in Section 1000.202. This Part incorporates no
	further amendments or editions to those objectives and conditions for operation.
(Sc	ource: Amended at 47 Ill. Reg, effective)

87 88 Section 1000.103 Scope 89 90 This The requirements of this Part applies apply to all persons who receive, possess, use, or 91 transfer material licensed under 10 CFR pursuant to Parts 30 through 35, 40, or 70, incorporated 92 by reference in Section 1000.202, or who are licensed to operate a production or utilization 93 facility underpursuant to 10 CFR 50, incorporated by reference in Section 1000.202 of the 94 regulations of the United States Nuclear Regulatory Commission. 95 96 (Source: Amended at 47 Ill. Reg. _____, effective _____) 97 98 SUBPART B: DEFINITIONS 99 100 **Section 1000.201 Definitions** 101 102 Except as stated in this Section, or unless a different meaning of a word or term is clear from the 103 context, the definition of words or terms in this Part are the same as that applied to the same 104 words or terms in the Environmental Protection Act [415 ILCS 5] As used in this Part: 105 106 "Act" means the Environmental Protection Act [415 ILCS 5, Ill. Rev. Stat., 1983, 107 ch. 11½, pars 1001 et seq.]. 108 109 "Board" means the Illinois Pollution Control Board. 110 111 "Department" means the Illinois Department of Nuclear Safety. 112 113 "Dose" means the quantity of radiation absorbed, per unit of mass, by the body or 114 by any portion of the body. Under this Part, When these regulations specify a dose during a period of time, the dose means the total quantity of radiation absorbed, 115 116 per unit of mass, by the body or by any portion of the body during such period of 117 time. The Several different units of dose are in current use. Definitions of units of doseas used in this Partthese regulations are set forth in the definitions of "Rad" 118 119 and "Rem", as defined in this Section. 120 121 "IEMA" means the Illinois Emergency Management Agency, Division of Nuclear 122 Safety. 123 124 "Individual" means any human being. 125 126 "Licensed activity" means any activity engaged in under a general or specific 127 license issued by the NRC. 128 129 "Licensed facility" means any facility constructed or operated under a permit or a

130 general or specific license issued by the NRC. 131 132 "Licensed material" means any material received, possessed, used, or transferred 133 under a general or specific license issued by the NRC. 134 135 "Licensee" means any person to whom a permit or a general or specific license 136 has been issued by the NRC. 137 "NRC" means the United States Nuclear Regulatory Commission. 138 139 140 "Rad" means a measure of the dose of any radiation to body tissues in terms of the 141 energy absorbed per unit mass of the tissue. One rad is the dose corresponding to 142 the absorption of 100 ergs per gram of tissue. (One millirad (mrad) = 0.001 rad). 143 144 "Radiation" means any or all of the following: alpha rays, beta rays, gamma rays, 145 X-rays, neutrons, high-speed highspeed electrons, high-speed protons, and other 146 atomic particles; but not sound or radio waves, or visible, infrared, or ultraviolet 147 light. 148 149 "Radioactive material" and "radioactive emissions" meanmeans any dusts, 150 particulates, fumes, mists, vapors, or gases which spontaneously emit ionizing 151 radiation. 152 153 "Rem" means a measure of the dose of any ionizing radiation to body tissue in 154 terms of its estimated biological effect relative to a dose received from an exposure to one roentgen of X-rays. (One millirem (mrem) = 0.001 rem). The 155 156 relation of rem to other dose units depends on upon the biological effect under consideration and upon the condition of irradiation. For the purpose of this Part, 157 158 any of the following is considered to be equivalent to a dose of one rem: 159 160 An exposure to one roentgen of X- or gamma radiation; 161 162 A dose of one rad due to X-, gamma, or beta radiation; 163 164 A dose of 0.1 rad due to neutrons or high energy protons; 165 166 A dose of 0.05 rad due to particles heavier than protons and with sufficient energy to reach the lens of the eye. If it is more convenient to measure the 167 neutron flux, or equivalent, than to determine the neutron dose in rads, one 168 169 rem of neutron radiation may for purposes of this Part be assumed to be 170 equivalent to 14 million neutrons per square centimeter incident upon the body; or, if there exists sufficient information is available to estimate with 171 172 reasonable accuracy the approximate distribution in the energy of

neutrons, the incident number of neutrons per square centimeter equivalent to one rem may be estimated from the following table.

Neutron Flux Dose Equivalents

	No. of N	eutron per	square	Average flux to deliver 100
Neutron Energy	centimet	er equivale	nt to a	millirem in 40 hours
(Mev)	dose of 1	rem (neutro	ns/cm ²)	(neutron/cm ² per second)
		,	ĺ	
Thermal		970×10^6		670
0.0001		720×10^6		500
0.005		820×10^6		570
0.02		400×10^6		280
0.1		120×10^6		80
0.5		43×10^6		30
1.0		26×10^6		18
2.5		29×10^6		20
5.0		26×10^6		18
7.5		24×10^6		17
10.0		24×10^6		17
10 to 30		14×10^6		10

"Restricted area" means any area <u>to which</u> access to which is controlled by the licensee <u>to protectfor purposes of protection of</u> individuals from exposure to radiation and radioactive materials. "Restricted area" <u>mustshall</u> not include any areas used as residential quarters, although a separate room or rooms in a residential building may be set apart as a restricted area.

"Unrestricted area" means any area <u>to which</u> access <u>to which</u> is not controlled by the licensee <u>to protect</u>, <u>for purposes of protection of</u> individuals from exposure to radiation and radioactive materials, and any area used for residential quarters.

(Source: Amended at 47 Ill. Reg. _____, effective _____)

Section 1000.202 Incorporations by Reference

The following materials are incorporated by reference. These incorporations by reference do not include any later amendments or editions:

a) Numerical Guides for Design Objectives and Limiting Conditions for Operations to Meet the Criterion "As Low as is Reasonably Achievable" for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents, 10 CFR 50, Appendix I (1984).

199 200	<u>b)</u>	Rules of General Applicability to Domestic Licensing of Byproduct Material, 10 CFR 30 (1984).
201		
202	<u>c)</u>	General Domestic Licenses for Byproduct Material, 10 CFR 31 (1984).
203 204	<u>d)</u>	Specific Domestic Licenses to Manufacture or Transfer Certain Items Containing
205	<u>37</u>	Byproduct Material, 10 CFR 32 (1984).
206		<u>= , </u>
207	<u>e)</u>	Specific Domestic Licenses of Broad Scope for Byproduct Material, 10 CFR 33
208		(1984).
209		
210	<u>f)</u>	Licenses for Industrial Radiography and Radiation Safety Requirements for
211	<u> </u>	Industrial Radiographic Operations, 10 CFR 34 (1984).
212		
213	<u>g)</u>	Medical Use of Byproduct Material, 10 CFR 35 (1984).
214		
215	<u>h)</u>	Domestic Licensing of Source Material, 10 CFR 40 (1984).
216		
217	<u>i)</u>	Domestic Licensing of Production and Utilization Facilities, 10 CFR 50 (1984).
218		
219	<u>j)</u>	Environmental Protection Regulations for Domestic Licensing and Related
220		Regulatory Functions, 10 CFR 51 (1984).
221		
222	<u>k)</u>	Domestic Licensing of Special Nuclear Material, 10 CFR 70 (1984).
223		
224	(Source	e: Added at 47 Ill. Reg, effective)
225		
226		SUBPART C: STANDARDS AND LIMITATIONS
227	G	
228	Section 1000.	301 Permissible Levels of Radiation in Unrestricted Areas
229	ANT	
230		nust notshall possess, use, receive, or transfer licensed material or engage in
231		ities in such manner in a way that creates as to create radiation levels in the air in
232	any unrestrict	ed area:
233	,	
234	a)	That could result in a dose to the whole body greater than 0.5 rem in any single
235		<u>year</u> Radiation levels in air such that any individual would be likely, when all
236		radioactive emissions by the licensee are <u>considered</u> taken into account, to receive
237		a dose to the whole body in excess of 0.5 rem in any one year;
238 239	b)	That could result in Dadiation levels in air which if an individual wars
239 240	b)	That could result in Radiation levels in air which, if an individual were continuously present in the area receiving a dose greater than 2 millirems in any
240 241		single hour, could result, when all radioactive emissions by the licensee are
·		THE REPORT OF THE PROPERTY OF

242		considered taken into account, in his receiving a dose in excess of 2 millirems in
243		any one hour; or
244		
245	c)	That could result in Radiation levels in air which, if an individual were
246		continuously present in the area receiving a dose greater than 100 millirems in
247		any 7 consecutive days, could result, when all radioactive emissions by the
248		licensee are considered taken into account, in his receiving a dose in excess of 100
249		millirems in any seven consecutive days.
250		
251	(Sour	rce: Amended at 47 Ill. Reg, effective)
252		
253	Section 1000	0.302 Radioactive Emissions to Unrestricted Areas
254	,	ANT
255	a)	ANo person must not shall possess, use, receive, or transfer licensed material or
256		engage in licensed activities in a way that releases so as to release to the air in an
257		unrestricted area radioactive material exceeding the concentration limits in
258		concentrations which exceed the limits specified in Appendix A-of this Part. For
259		purposes of this Section, concentrations of radioactive material may be averaged
260		over a period not greater than one year.
261 262	b)	For the number of this Section section the concentration limits in Annuality A of
262 263	b)	For the purpose of this Section, section the concentration limits in Appendix A of this Part shall apply at the boundary of the restricted area. The concentration of
263 264		radioactive material discharged through a stack, pipe or similar conduit may be
265		determined for with respect to the point where the material leaves the conduit. If
266		the conduit discharges within the restricted area, the concentration at the boundar
267		may be determined by applying established factors for dilution, dispersion, or
268		decay between the point of discharge and the boundary.
269		decay seemed the point of discharge and the countary.
270	(Sour	rce: Amended at 47 Ill. Reg, effective)
271	`	<u> </u>
272		SUBPART D: ADDITIONAL REQUIREMENTS
273		
274	Section 1000	0.401 Applicability
275		
276	This Subpart	applies The provisions of this part apply to radiation doses received by members of
277	-	the general environment and to radioactive materials introduced into the general
278	environment	<u>due to</u> as the result of operations that which are part of a nuclear fuel cycle.
279		
280	(Sour	rce: Amended at 47 Ill. Reg, effective)
281		
282	Section 1000	0.402 Definitions
283		
284	As used in th	us Subpart:

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"Curie" (Ci) means the that quantity of radioactive material that produces producing 37 billion nuclear transformations per second. (One millicurie (mCi)=0.001 Ci.)

"Dose equivalent" means the product of absorbed dose and appropriate factors to account for <u>differences</u> in biological effectiveness due to the quality of radiation and its spatial distribution in the body. The unit of dose equivalent is the "rem." (One millirem (mrem) = 0.001 rem.)

"General environment" means the total terrestrial, atmospheric, and aquatic environments outside sites upon which any operation <u>that which</u> is part of a nuclear fuel cycle is conducted.

"Gigawatt-year" refers to the quantity of electrical energy produced at the busbar of a generating station. A gigawatt is equal to one billion watts. A gigawatt-year is equivalent to the amount of energy output represented by an average electric power level of one gigawatt sustained for one year.

"Member of the public" means any individual whothat can receive a radiation dose in the general environment, whether or not the personhe is may or may not also be exposed to radiation in an occupation associated with a nuclear fuel cycle. However, an individual is not considered a member of the public during any period in which that individualhe is engaged in carrying out any operation that which is part of a nuclear fuel cycle.

"Nuclear fuel cycle" means the operations defined to be associated with the production of electrical power for public use by any fuel cycle through utilization of nuclear energy.

"Organ" means any human organ exclusive of the dermis, the epidermis, or the cornea.

"Site" means the area contained within the boundary of a location under the control of persons possessing or using radioactive material on which is conducted one or more operations covered by this <u>Part is conducted</u> part.

"Uranium fuel cycle" means the operations of milling of uranium ore, chemical conversion of uranium, isotopic enrichment of uranium, fabrication of uranium fuel, generation of electricity by a light-water-cooled nuclear power plant using uranium fuel, and reprocessing of spent uranium fuel, to the extent that these directly support the production of electrical power for public use utilizing nuclear energy.; "Uranium fuel cycle" but excludes mining operations, operations at waste

328		disposal sites, transportation of any radioactive material in support of these
329		operations, and the reuse of recovered <u>non-uranium</u> special nuclear
330		and byproductby-product materials from the cycle.
331		
332	(Sou	rce: Amended at 47 Ill. Reg, effective)
333		
334	Section 100	0.403 Environmental Standards for Uranium Fuel Cycle
335		
336	A person co	nducting operations Operations covered by this Subpart must conduct them in a way
337	that provides	sshall be conducted in such a manner as to provide reasonable assurance that:
338		
339	a)	The annual dose equivalent does not exceed 25 millirems to the whole body, 75
340		millirems to the thyroid, and 25 millirems to any other organ of any member of
341		the public as the result of exposures to planned discharges of radioactive
342		materials, radon and its daughters excepted, to the general environment from
343		uranium fuel cycle operations, and to radiation from these operations.
344		
345	b)	The total quantity of radioactive materials entering the general environment from
346		the entire uranium fuel cycle, per gigawatt-year of electrical energy produced by
347		the fuel cycle, contains less than 50,000 curies of krypton-85, 5 millicuries of
348		iodine-129, and 0.5 millicuries combined of plutonium-239 and other alpha-
349		emitting transuranic radionuclides with the half-liveshalflives greater than one
350		year.
351		
352	(Sou	rce: Amended at 47 Ill. Reg, effective)
353		
354		SUBPART E: RECORDS
355		
356	Section 100	0.501 Records
357		
358	A person All	persons subject to this Part mustshall submit to IEMAthe Department, forwith
359	respect to an	y material or facility permitted or licensed by the NRC or for which an NRC permit
360	or license is	sought:
361		
362	a)	Preliminary Safety Analysis Report and Final Safety Analysis Report, as
363	,	described in 10 CFR 50.34, incorporated by reference in Section 1000.202.
364		
365	b)	Application for Construction Permit and for all amendments to that permit
366	,	thereto, including information required by 10 CFR 50.34a, 50.36, and 51.20,
367		incorporated by reference in Section 1000.202.
368		
369	c)	Environmental Impact Appraisal, Draft and Final Environmental Impact
370	,	Statement. Negative Declaration, or other document prepared by the NRC under

371		10 CFR 51, incorporated by reference in Section 1000.202.
372		
373	d)	Operating Permit and all amendments to that permit thereto, including Technical
374		Specifications under 10 CFR 50.36a, incorporated by reference in Section
375		<u>1000.202</u> .
376		
377	e)	Application for Amendment to Operating License.
378		
379	f)	All data, records, and reports conducted by or for that person and submitted to the
380		NRC <u>for</u> in connection with determining or predicting radiation levels in <u>the</u> air in
381		unrestricted areas or the type or amount of radioactive materials emitted into the
382		air conducted by or for such persons.
383		
384	(Sour	ce: Amended at 47 Ill. Reg, effective)
385		
386	Section 1000	.502 Notification of Incidents
387		
388		subject to this Part <u>mustshall</u> immediately notify <u>IEMA</u> by telephone and telegraph,
389	<u> </u>	facsimile, the Manager of the Office of Nuclear Facility Safety of the Illinois
390	*	of Nuclear Safety, 1035 Outer Park Drive, Springfield, Illinois 62704, of any
391		ondition arising from the use or possession of licensed materials or facilities or the
392	_	f licensed activities which may have caused or threatens to cause emissions or
393		els exceedingin excess of those allowed under this Part. <u>IEMA's 24-hour</u>
394	Operations C	enter can be reached for notification of incidents at 1-217-782-7860.
395		
396	(Sour	ce: Amended at 47 Ill. Reg, effective)
397		
398	Section 1000	.503 Other Provisions
399		
400	a)	The definitions specified set out in 35 Ill. Adm. Code 201.102 apply to this Part.
401		
102	b)	All persons subject to this Part are subject to the requirements and provisions of
103		35 Ill. Adm. Code 201.122, 201.123, 201.124, 201.125, 201.126, 201.141,
104		201.150 and 201.151.
405		
106	(Sour	ce: Amended at 47 Ill. Reg, effective)
107		

Section 1000.APPENDIX A Concentrations in Air Above Natural Background

408

409

Element (atomic number)	Isotope ¹		$\mu \text{Ci/ml}$
Actinium (89)	AC227	<u>S</u>	\$ 8 x 10 ⁻¹⁴
	<u>AC 228</u>	<u>I</u> <u>S</u> <u>I</u>	19 x 10 ⁻¹³ AC 228S 3 x 10 ⁻⁹ 16 x 10 ⁻¹⁰
Americium (95)	Am 241 <mark>\$</mark>	<u>S</u> I	2 x 10 ⁻¹³ 14 x 10 ⁻¹²
	<u>Am 242m</u>	<u>S</u>	Am 242mS2 x 10 ⁻¹³ 19 x 10 ⁻¹²
	<u>Am 242</u>	<u>S</u>	Am 242S 1 x 10 ⁻⁹ 12 x 10 ⁻⁹
	<u>Am 243</u>	<u>I</u> <u>S</u>	Am 243S2 x 10 ⁻¹³ 14 x 10 ⁻¹²
	<u>Am 244</u>	<u>I</u> <u>S</u>	Am 244S1 x 10 ⁻⁷
Antimony	Sb 122	$\frac{1}{S}$	18 x 10 ⁻⁷ \$6 x 10 ⁻⁹
	<u>Sb 124</u>	<u>I</u> <u>S</u>	I5 x 10 ⁻⁹ Sb 124S5 x 10 ⁻⁹
	<u>Sb 125</u>	<u>I</u> <u>S</u>	From 10 10 10 10 10 10 10 10 10 10 10 10 10
Argon (18)	A 37	<u>I</u> Sub ²	I9 x 10 ⁻¹⁰ Sub ² 1 x 10 ⁻⁴
	<u>A 41</u>	<u>Sub</u> <u>S</u> <u>I</u> <u>S</u> <u>I</u> <u>S</u> <u>I</u> <u>S</u> <u>I</u> <u>S</u> <u>I</u> <u>S</u>	$\frac{A \cdot 41Sub}{2} \times 10^{-8}$
Arsenic (33)	As 73	<u>S</u>	$\frac{$}{$}$ 7 x 10 ⁻⁸
	A = 74	<u>I</u>	I1 x 10 ⁻⁸
	<u>As 74</u>	7	As 74S1 x 10 ⁻⁸ 14 x 10 ⁻⁹
	As 76	<u>r</u>	As 76S4 x 10 ⁻⁹
	<u>AS 70</u>	<u>ร</u>	I3 x 10 ⁻⁹
	<u>As 77</u>	S	$\frac{15 \times 10^{-8}}{4 \times 778} \times 10^{-8}$
	115 //	_	I1 x 10 ⁻⁸
Astatine (85)	At 211	$\frac{1}{S}$	2×10^{-10}
		Ī	1 x 10 ⁻⁹
Barium (56)	Ba 131	S	4 x 10 ⁻⁸
,		I	1 x 10 ⁻⁸
	Ba 140		Ba 140S 4 x 10 ⁻⁹
		<u>S</u> I	1 x 10 ⁻⁹
Berkelium (97)	Bk 249	S	3 x 10 ⁻¹¹
. ,		I	4 x 10 ⁻⁹
	Bk 250	S	5 x 10 ⁻⁹

		I	4 x 10 ⁻⁸
Berylium (4)	Be 7	S	2×10^{-7}
Berynum (+)	DC /	I	4 x 10 ⁻⁸
Bismuth (83)	Bi 206	S	6 x 10 ⁻⁹
Dismuin (83)	21200	I	5×10^{-9}
	Bi 207	S	6×10^{-9}
		I	5 x 10 ⁻¹⁰
	Bi 210	S	2 x 10 ⁻¹⁰
		I	2 x 10 ⁻¹⁰
	Bi 212	S	3×10^{-9}
		I	7×10^{-9}
Bromine (35)	Br 82	S	4×10^{-8}
		I	6 x 10 ⁻⁹
Cadmium (48)	Cd 109	S	2×10^{-9}
		I	3×10^{-9}
	Cd 115m	S	1×10^{-9}
		I	1×10^{-9}
	Cd 115	S	8×10^{-9}
		I	6×10^{-9}
Calcium (20)	Ca 45	S	1 x 10 ⁻⁹
		I	4 x 10 ⁻⁹
	Ca 47	S	6×10^{-9}
		I	6×10^{-9}
Californium (98)	Cf 249	S	5 x 10 ⁻¹⁴
		I	3×10^{-12}
	Cf 250	S	2×10^{-13}
		I	3×10^{-12}
	Cf 251	S	6 x 10 ⁻¹⁴
		I	3×10^{-12}
	Cf 252	S	2×10^{-13}
	CC 252	I	1×10^{-12}
	Cf 253	S	3×10^{-11}
	CC 25.4	I	3×10^{-11}
	Cf 254	S	2×10^{-13}
Carbon (6)	C 14	I S	2 x 10 ⁻¹³ 1 x 10 ⁻⁷
Carbon (6)	(CO_2)	Sub	1 x 10 ⁻⁶
Cerium (58)	Ce 141	Sub	2×10^{-8}
Corruin (50)	CC 141	I	5×10^{-9}
	Ce 143	S	9×10^{-9}
	CC 173	I	7×10^{-9}
	Ce 144	S	3×10^{-10}
	00111	I	2×10^{-10}
		-	- 11 10

Cesium (55)	Cs 131	S	4 x 10 ⁻⁷ 1 x 10 ⁻⁷
	Cs 134m	I S	1×10^{-6}
	CS 154III	I	2×10^{-7}
	Cs 134	S	1×10^{-9}
	C5 15 1	I	4×10^{-10}
	Cs 135	S	2×10^{-8}
		Ī	3×10^{-9}
	Cs 136	S	1 x 10 ⁻⁸
		I	6 x 10 ⁻⁹
	Cs 137	S	2×10^{-9}
		I	5×10^{-10}
Chlorine (17)	C1 36	S	1×10^{-8}
	G1 20	I	8×10^{-10}
	C1 38	S	9×10^{-8}
	G 51	I	7×10^{-8}
Chromium (24)	Cr 51	S	4×10^{-7}
C 1 1 (27)	0.57	I	8×10^{-8}
Cobalt (27)	Co 57	S	1×10^{-7}
	C - 50	I	6×10^{-9}
	Co 58m	S	6×10^{-7}
	Co 50	I	3 x 10 ⁻⁷ 3 x 10 ⁻⁸
	Co 58	S I	2×10^{-9}
	Co 60	S	2 x 10 1 x 10 ⁻⁸
	C0 00	I	3×10^{-10}
Copper (29)	Cu 64	S	7×10^{-8}
Copper (2))	Cu 0+	I	4×10^{-8}
Curium (96)	Cm 242	S	4×10^{-12}
Curium (90)	CIII 242	I	6×10^{-12}
	Cm 243	S	2×10^{-13}
	CIII 2 18	I	3×10^{-12}
	Cm 244	S	3×10^{-13}
		I	3 x 10 ⁻¹²
	Cm 245	S	2×10^{-13}
		I	4 x 10 ⁻¹²
	Cm 246	S	2×10^{-13}
		I	4×10^{-12}
	Cm 247	S	2×10^{-13}
		I	4 x 10 ⁻¹²
	Cm 248	S	2 x 10 ⁻¹⁴
		I	4×10^{-13}
	Cm 249	S	4×10^{-7}

		I	4×10^{-7}
Dysprosium (66)	Dy 165	S	9×10^{-8}
		I	7 x 10 ⁻⁸
	Dy 166	S	8 x 10 ⁻⁹
	•	I	7 x 10 ⁻⁹
Einsteinium (99)	Es 253	S	3 x 10 ⁻¹¹
		I	2 x 10 ⁻¹¹
	Es 254m	S	2×10^{-10}
		I	2 x 10 ⁻¹⁰
	Es 254	S	6 x 10 ⁻¹³
		I	4 x 10 ⁻¹²
	Es 255	S	2 x 10 ⁻¹¹
	25 200	Ĭ	1 x 10 ⁻¹¹
Erbium (68)	Er 169	S	2×10^{-8}
Ziolam (00)	Li 10)	I	1 x 10 ⁻⁸
	Er 171	S	2 x 10 ⁻⁸
	L1 1/1	I	2×10^{-8}
Europium (63)	Eu 152	S	1 x 10 ⁻⁸
Europium (03)	(T/2=9.2 hrs)	I	1 x 10 ⁻⁸
	Eu 152	S	4×10^{-10}
	(T/2=13 yrs)	I	6×10^{-10}
	Eu 154	S	1 x 10 ⁻¹⁰
	Eu 134	S I	2×10^{-10}
	Ev. 155		2 x 10
	Eu 155	S	3 x 10 ⁻⁹
F(100)	E. 254	I	3×10^{-9}
Fermium (100)	Fm 254	S	2×10^{-9}
	E. 255	I	2×10^{-9}
	Fm 255	S	6 x 10 ⁻¹⁰
	F 056	I	4×10^{-10}
	Fm 256	S	1 x 10 ⁻¹⁰
TI (0)	F 10	I	6 x 10 ⁻¹¹
Fluorine (9)	F 18	S	2×10^{-7}
	C 1 1 7 0	I	9×10^{-8}
Gadolinium (64)	Gd 153	S	8×10^{-9}
		I	3×10^{-9}
	Gd 159	S	2×10^{-8}
		I	1×10^{-8}
Gallium (31)	Ga 72	S	8×10^{-9}
		I	6 x 10 ⁻⁹
Germanium (32)	Ge 71	S	4×10^{-7}
		I	2×10^{-7}
Gold (79)	Au 196	S	4×10^{-8}
		I	2×10^{-8}

	Au 198	S	1×10^{-8}
		I	8×10^{-9}
	Au 199	S	4×10^{-8}
		I	3×10^{-8}
Hafnium (72)	Hf 181	S	1×10^{-9}
		I	3×10^{-9}
Holmium (67)	Но 166	S	$7x\ 10^{-9}$
		I	6×10^{-9}
Hydrogen (1)	Н3	S	2×10^{-7}
		I	$2x10^{-7}$
		Sub	4×10^{-5}
Indium (49)	In 113m	S	3×10^{-7}
		I	2×10^{-7}
	In 114m	S	4×10^{-9}
		I	7×10^{-10}
	In 115m	S	8×10^{-8}
		I	6×10^{-8}
	In 115	S	9×10^{-9}
		I	1 x 10 ⁻⁹
Iodine (53)	I 125	S	8 x 10 ⁻¹¹
		I	6 x 10 ⁻⁹
	I 126	S	9 x 10 ⁻¹¹
		I	1 x 10 ⁻⁸
	I 129	S	2 x 10 ⁻¹¹
		I	2 x 10 ⁻⁹
	I 131	S	1 x 10 ⁻¹⁰
		I	1 x 10 ⁻⁸
	I 132	S	3 x 10 ⁻⁹
		I	3×10^{-8}
	I 133	S	4×10^{-10}
		I	7 x 10 ⁻⁹
	I 134	S	6 x 10 ⁻⁹
		I	1×10^{-7}
	I 135	S	1 x 10 ⁻⁹
		I	1 x 10 ⁻⁸
Iridium (77)	Ir 190	S	4×10^{-8}
, ,		I	1 x 10 ⁻⁸
	Ir 192	S	4×10^{-9}
		I	9 x 10 ⁻¹⁰
	Ir 194	S	8 x 10 ⁻⁹
		I	5 x 10 ⁻⁹
Iron (26)	Fe 55	S	3×10^{-8}
		I	3 x 10 ⁻⁸

	Fe 59	S I	5 x 10 ⁻⁹ 2 x 10 ⁻⁹
Krypton (36)	Kr 85m	Sub	$1x\ 10^{-7}$
, ,	Kr 85	Sub	3×10^{-7}
	Kr 87	Sub	2×10^{-8}
	Kr 88	Sub	2×10^{-8}
Lanthanum (57)	La 140	S	5x 10 ⁻⁹
		I	4 x 10 ⁻⁹
Lead (82)	Pb 203	S	9x 10 ⁻⁸
		I	6×10^{-8}
	Pb 210	S	$4x \ 10^{-12}$
		I	8×10^{-12}
	Pb 212	S	6×10^{-10}
		I	7×10^{-10}
Lutetium (71)	Lu 177	S	2×10^{-8}
		I	2×10^{-8}
Manganese (25)	Mn 52	S	7 x 10 ⁻⁹
		I	5 x 10 ⁻⁹
	Mn 54	S	1×10^{-8}
		I	1×10^{-9}
	Mn 56	S	3×10^{-8}
		I	2×10^{-8}
Mercury (80)	Hg 197m	S	3×10^{-8}
		I	3×10^{-8}
	Hg 197	S	4×10^{-8}
		I	9×10^{-8}
	Hg 203	S	2×10^{-9}
		I	4×10^{-9}
Molybdenum (42)	Mo 99	S	3×10^{-8}
		I	7×10^{-9}
Neodymium (60)	Nd 144	S	3×10^{-12}
		I	1×10^{-11}
	Nd 147	S	1×10^{-8}
		I	8×10^{-9}
	Nd 149	S	6×10^{-8}
		I	5×10^{-8}
Neptunium (93)	Np 237	S	1×10^{-13}
		I	4×10^{-12}
	Np 239	S	3×10^{-8}
	371.70	I	2×10^{-8}
Nickel (28)	Ni 59	S	2×10^{-8}
	N. 60	I	3×10^{-8}
	Ni 63	S	2×10^{-9}

		I	1 x 10 ⁻⁸
	N: 65		3×10^{-8}
	Ni 65	S	
NT. 1.	NI 02	I	2×10^{-8}
Niobium	Nb 93m	S	4 x 10 ⁻⁹
(Columbium)(41)		.	5 10-9
	NH 05	I	5×10^{-9}
	Nb 95	S	2×10^{-8}
		I	3×10^{-9}
	Nb 97	S	2×10^{-7}
		I	2×10^{-7}
Osmium (76)	Os 185	S	2×10^{-8}
		I	2 x 10 ⁻⁹
	Os 191m	S	6×10^{-7}
		I	3×10^{-7}
	Os 191	S	4×10^{-8}
		I	1×10^{-8}
	Os 193	S	1 x 10 ⁻⁸
		I	9 x 10 ⁻⁹
Palladium (46)	Pd 103	S	5×10^{-8}
		I	3×10^{-8}
	Pd 109	S	2×10^{-8}
		I	1 x 10 ⁻⁸
Phosphorus (15)	P 32	S	2 x 10 ⁻⁹
1		I	3 x 10 ⁻⁹
Platinum (78)	Pt 191	S	3 x 10 ⁻⁸
(, 0)	- 1 - 1 -	Ī	2 x 10 ⁻⁸
	Pt 193m	S	2×10^{-7}
	1 (1) (1) (1)	Ĭ	2×10^{-7}
	Pt 193	S	4 x 10 ⁻⁸
	11175	I	1 x 10 ⁻⁸
	Pt 197m	S	2×10^{-7}
	1117/111	I	2×10^{-7}
	Pt 197	S	3×10^{-8}
	1(1)/	I	2×10^{-8}
Plutonium (94)	Pu 238	S	7×10^{-14}
Tutomum (54)	1 u 250	I	1 x 10 ⁻¹²
	Pu 239	S	6 x 10 ⁻¹⁴
	1 u 239	I	1×10^{-12}
	Pu 240	S	6 x 10 ⁻¹⁴
	1 u 440		U X 1U 1 v 10-12
	Do 241	I	1×10^{-12}
	Pu 241	S	3×10^{-12}
	D 242	I	1×10^{-9}
	Pu 242	S	6 x 10 ⁻¹⁴

		I	1 x 10 ⁻¹²
	Pu 243	S	6 x 10 ⁻⁸
	1 u 2 -1 3	I	8 x 10 ⁻⁸
	Pu 244	S	6 x 10 ⁻¹⁴
	1 u 2++	I	1×10^{-12}
Polonium (84)	Po 210	S	2 x 10 ⁻¹¹
1 Olollium (04)	10210	I	7×10^{-12}
Potassium (19)	K 42	S	7×10^{-8}
1 Otassium (19)	N 42	I	4 x 10 ⁻⁹
Praseodymium (59)	Pr 142	S	7×10^{-9}
riaseodynnum (39)	F1 142	I	5 x 10 ⁻⁹
	Pr 143	S	1 x 10 ⁻⁸
	F1 143	I	6 x 10 ⁻⁹
Promothium (61)	Dm 147	S	2×10^{-9}
Promethium (61)	Pm 147	I	2 x 10-9
	Dm 140		3 x 10 ⁻⁹ 1 x 10 ⁻⁸
	Pm 149	S	1 X 10 °
Durate a stinierum (01)	D- 220	I	8×10^{-9}
Protoactinium (91)	Pa 230	S	6×10^{-11}
	D 021	I	3×10^{-11}
	Pa 231	S	4×10^{-14}
	D 222	I	4×10^{-12}
	Pa 233	S	2×10^{-8}
D 11 (00)	D 000	I	6×10^{-9}
Radium (88)	Ra 223	S	6×10^{-11}
		I	8×10^{-12}
	Ra 224	S	2×10^{-10}
		I	2×10^{-11}
	Ra 226	S	3×10^{-12}
		I	2×10^{-12}
	Ra 228	S	2×10^{-12}
		I	1×10^{-12}
Radon (86)	Rn 220	S	1 x 10 ⁻⁸
	Rn 222 ³	3×10^{-9}	3×10^{-9}
Rhenium (75)	Re 183	S	9×10^{-8}
		I	5×10^{-9}
	Re 186	S	2×10^{-8}
		I	8 x 10 ⁻⁹
	Re 187	S	3×10^{-7}
		I	2×10^{-8}
	Re 188	S	1 x 10 ⁻⁸
		I	6 x 10 ⁻⁹
Rhodium (45)	Rh 103m	S	3×10^{-6}
-		I	2×10^{-6}

	Rh 105	S	3×10^{-8}
D 1:1: (27)	D1 06	I	2×10^{-8}
Rubidium (37)	Rb 86	S	1×10^{-8}
	D1 07	I	2×10^{-9}
	Rb 87	S	2×10^{-8}
5 4	D 0.5	I	2×10^{-9}
Ruthenium (44)	Ru 97	S	8×10^{-8}
	5 400	I	6 x 10 ⁻⁸
	Ru 103	S	2×10^{-8}
		I	3×10^{-9}
	Ru 105	S	2×10^{-8}
		I	2×10^{-8}
	Ru 106	S	3×10^{-9}
		I	2×10^{-10}
Samarium (62)	Sm 147	S	2 x 10 ⁻¹²
		I	9×10^{-12}
	Sm 151	S	2×10^{-9}
		I	5 x 10 ⁻⁹
	Sm 153	S	2×10^{-8}
		I	1×10^{-8}
Scandium (21)	Sc 46	S	8 x 10 ⁻⁹
		I	8 x 10 ⁻¹⁰
	Sc 47	S	2 x 10 ⁻⁸
		I	2 x 10 ⁻⁸
	Sc 48	S	6 x 10 ⁻⁹
		I	5 x 10 ⁻⁹
Selenium (34)	Se 75	S	4 x 10 ⁻⁸
(-)		I	4 x 10 ⁻⁹
Silicon (14)	Si 31	S	2 x 10 ⁻⁷
		Ĭ	3×10^{-8}
Silver (47)	Ag 105	S	2×10^{-8}
	116 100	I	3×10^{-9}
	Ag 110m	S	7×10^{-9}
	ng mom	I	3×10^{-10}
	Ag 111	S	1 x 10 ⁻⁸
	Ag III	I	8 x 10 ⁻⁹
Sodium (11)	Na 22	S	6 x 10 ⁻⁹
Soulum (11)	1 va 22	I	3×10^{-10}
	Na 24	S	4 x 10 ⁻⁸
	1 1 a 4 4	S I	5 x 10 ⁻⁹
Strontium (38)	Sr 95m	S	1×10^{-6}
Suomum (56)	Sr 85m	S I	1 x 10 1 x 10 ⁻⁶
	Cr. 05		
	Sr 85	S	8 x 10 ⁻⁹

	Sr 89	I S	4 x 10 ⁻⁹ 3 x 10 ⁻¹⁰
	51 09	I	1 x 10 ⁻⁹
	Sr 90	S	3×10^{-11}
	51 70	I	2×10^{-10}
	Sr 91	S	2×10^{-8}
	51 71	I	9×10^{-9}
	Sr 92	S	2×10^{-8}
	21,72	Ĭ	1 x 10 ⁻⁸
Sulfur (16)	S 35	S	9×10^{-9}
2 332 32 (2 3)		Ĩ	9×10^{-9}
Tantalum (73)	Ta 182	S	1 x 10 ⁻⁹
` ,		I	7 x 10 ⁻¹⁰
Technetium (43)	Tc 96m	S	3×10^{-6}
, ,		I	1 x 10 ⁻⁶
	Tc 96	S	2×10^{-8}
		I	8 x 10 ⁻⁹
	Tc 97m	S	8 x 10 ⁻⁸
		I	5 x 10 ⁻⁹
	Tc 97	S	4×10^{-7}
		I	1×10^{-8}
	Tc 99m	S	1×10^{-6}
		I	5×10^{-7}
	Tc 99	S	7×10^{-8}
		I	2×10^{-9}
Tellurium (52)	Te 125m	S	1×10^{-8}
		I	4×10^{-9}
	Te 127m	S	5×10^{-9}
		I	1×10^{-9}
	Te 127	S	6×10^{-8}
		I	3×10^{-8}
	Te 129m	S	3×10^{-9}
		I	1×10^{-9}
	Te 129	S	2×10^{-7}
		I	1×10^{-7}
	Te 131m	S	1×10^{-8}
	T	I	6×10^{-9}
	Te 132	S	7×10^{-9}
T 1. (55)	FFI 160	I	4×10^{-9}
Terbium (65)	Tb 160	S	3×10^{-9}
TT 11: (01)	TTI 200	I	1×10^{-9}
Thallium (81)	Tl 200	S	9×10^{-8}
		I	4 x 10 ⁻⁸

	Tl 201	S I	7 x 10 ⁻⁸ 3 x 10 ⁻⁸
	Tl 202	S	3×10^{-8}
		I	8×10^{-9}
	Tl 204	S	2×10^{-8}
- (OO)	FI 225	I	9×10^{-10}
Thorium (90)	Th 227	S	1×10^{-11}
	TTI 220	I	6×10^{-12}
	Th 228	S	3×10^{-13}
	TTI 220	I	2×10^{-13}
	Th 230	S	8 x 10 ⁻¹⁴
	Th 221	I S	3 x 10 ⁻¹³ 5 x 10 ⁻⁸
	Th 231	S I	4×10^{-8}
	Th 232	S	1×10^{-12}
	111 232	I	1 x 10 ⁻¹²
	Th natural	S	2×10^{-12}
	I II IIuturur	I	2×10^{-12}
	Th 234	S	2×10^{-9}
		Ĭ	1×10^{-9}
Thulium (69)	Tm 170	S	1 x 10 ⁻⁹
` ,		I	1 x 10 ⁻⁹
	Tm 171	S	4 x 10 ⁻⁹
		I	8 x 10 ⁻⁹
Tin (50)	Sn 113	S	1×10^{-8}
		I	2×10^{-9}
	Sn 125	S	4×10^{-9}
		I	3×10^{-9}
Tungsten (Wolfram) (74)	W 181	S	8×10^{-8}
	W. 405	I	4×10^{-9}
	W 185	S	3×10^{-8}
	W 107	I	4×10^{-9}
	W 187	S	2 x 10 ⁻⁸
Hranium (02)	11 220	I	1 x 10 ⁻⁸ 1 x 10 ⁻¹¹
Uranium (92)	U 230	S I	4×10^{-12}
	U 232	S	3×10^{-12}
	0 232	I	9×10^{-13}
	U 233	S	2×10^{-11}
	C 200	I	4×10^{-12}
	U 234	S^4	2 x 10 ⁻¹¹
		Ī	4 x 10 ⁻¹²
	U 235	S^4	2 x 10 ⁻¹¹

		т	4 x 10 ⁻¹²
	U 236	I S	2 x 10 ⁻¹¹
	0 230	I	4×10^{-12}
	U 238	S^4	3×10^{-12}
	0 236	I	5×10^{-12}
	U 240	S	8×10^{-9}
	0 240	I	6 x 10 ⁻⁹
	U-natural	S^4	5×10^{-12}
	Chatarar	I	5×10^{-12}
Vanadium (23)	V 48	S	6×10^{-9}
vanadram (23)	V 10	I	2×10^{-9}
Xenon (54)	Xe 131m	Sub	4×10^{-7}
11011011 (0 1)	Xe 133	Sub	3×10^{-7}
	Xe 133m	Sub	3×10^{-7}
	Xe 135	Sub	1×10^{-7}
Ytterbium (70)	Yb 175	S	2×10^{-8}
(1.2)		I	2 x 10 ⁻⁸
Yttrium (39)	Y 90	S	4 x 10 ⁻⁹
		I	3 x 10 ⁻⁹
	Y 91m	S	8×10^{-7}
		I	6 x 10 ⁻⁷
	Y 91	S	1 x 10 ⁻⁹
		I	1 x 10 ⁻⁹
	Y 92	S	1 x 10 ⁻⁸
		I	1 x 10 ⁻⁸
	Y 93	S	6 x 10 ⁻⁹
		I	5 x 10 ⁻⁹
Zinc (30)	Zn 65	S	4 x 10 ⁻⁹
		I	2×10^{-9}
	Zn 69m	S	1×10^{-8}
		I	1×10^{-8}
	Zn 69	S	2×10^{-7}
		I	3×10^{-7}
Zirconium (40)	Zr 93	S	4×10^{-9}
		I	1×10^{-8}
	Zr 95	S	4×10^{-9}
		Ι	1×10^{-9}
	Zr 97	S	4×10^{-9}
		I	3×10^{-9}
Any single radionuclide not listed above with decay mode other than alpha emission or		Sub	3 x 10 ⁻⁶

spontaneous fission and with <u>radioactive</u> radioactive half-life less than 2 hours.

1 x 10⁻¹⁰

Any single radionuclide not listed above with decay mode other than alpha emission or spontaneous fission and with <u>radioactiveradioactive</u> half-life greater than 2 hours.

2 x 10⁻¹⁴

Any single radionuclide not listed above, that which decays by alpha emission or spontaneous fission.

¹Soluble (S); Insoluble (I).

²"Sub" means that values given are for submersion in a semispherical infinite cloud of airborne material.

³These radon concentrations are appropriate for protection from radon-222 combined with its short-lived daughters. The value may be replaced by one-thirtieth (1/30) of a "working level." (A "working level" is defined as any combination of short-lived radon-222 daughters, polonium-218, lead-214, bismuth-214 and polonium-214, in one liter of air, without regard to the degree of equilibrium, that will result in the ultimate emission of 1.3 x 10⁵ MeV of alpha particle energy.

⁴For soluble mixtures of U-238, U-234 and U-235 in air chemical toxicity may be the limiting factor. The concentration value is 0.007 milligrams of uranium per cubic meter of air. The specific activity for natural uranium is 6.77 x 10⁻⁷ curies per gram U. The specific activity (SA) for other mixtures of U-238, U-235 and U-234, if not known, willshall be:

```
SA=3.6 \ x \ 10^{-7} \ curies/gram \ U ...... U-depleted SA=(0.4+0.38 \ E+0.0034 \ E^2) \ 10^{-6}.... E \ge 0.72
```

where E is the percentage by weight of U-235, expressed as a percent.

 NOTE: When In any case where there is a mixture in air of more than one radionuclide exists, the limiting values for purposes of this Appendix should be determined as follows:

1. If the identity and concentration of each radionuclide in the mixture are known, the limiting values should be derived as follows: Determine, for each radionuclide in the mixture, the ratio between the quantity present in the mixture and the limit otherwise established in Appendix A for the specific radionuclide when not in a mixture. The sum of thesuch ratios for all the radionuclides in the mixture may not exceed "1" (i.e., "unity").

EXAMPLE: If radionuclides A, B, and C are present in concentrations C_A , C_B , C_C , and if the applicable $\underline{\mathsf{MPC}}$'s are MPC A, and MPC B, and MPC C respectively, then the concentrations $\underline{\mathsf{must}}$ be limited so that the following relationship exists:

$$(C_A/MPC_A) + (C_B/MPC_B) + (C_C/MPC_C) \le 1$$

- 2. If either the identity or the concentration of any radionuclide in the mixture is not known, the limiting values for purposes of Appendix A mustshall be 2 x 10⁻¹⁴.
- 3. If any of the conditions specified below are met, the corresponding values specified below may be used in <u>insteadlieu</u> of those specified in paragraph 2 above.
 - a. If the identity of each radionuclide in the mixture is known but the concentration of one or more of the radionuclides in the mixture is not known, the concentration limit for the mixture is the limit specified in Appendix A for the radionuclide in the mixture having the lowest concentration limit; or
 - b. If the identity of each radionuclide in the mixture is <u>not</u>now known, but it is known that <u>certain</u> radionuclides specified in Appendix A are not present in the mixture, the concentration limit for the mixture is the lowest concentration limit specified in Appendix A for any radionuclide <u>that</u>which is not known to be absent from the mixture; or
 - c. Element (atomic number) and isotope, µCi/ml

If it is known that alpha-emitters and Sr 90, I 129, Pb 210, Ac 1 x 10⁻¹⁰ 227, Ra 228, Pa 230, Pu 241, and Bk are not present.

If it is known that alpha-emitters and Pb 210, Ac 227, Ra 228, 1×10^{-11} and Pu 241 are not present.

If it is known that alpha-emitters and Ac 227 are not present. 1×10^{-12}

If it is known that Ac 227, Th 230, Pa 231, Pu 238, Pu 239, Pu 1 x 10⁻¹³ 240, Pu 242, Pu 244, Cm 248, Cf 249 and Cf 251 are not present.

469		
470	4.	If a mixture of radionuclides consists of uranium and its daughters in ore dust before prior
471		to chemical separation of the uranium from the ore, the following values may be used for
472		uranium and its daughters through radium-226, instead of those from paragraphs 1, 2, or
473		3 above:
474		
475		3 x 10-12 μCi/ml gross alpha activity; 2 x 10-12 μCi/ml natural uranium; or 3
476		micrograms per cubic meter of air natural uranium.
477		
478	5.	For purposes of this note, a radionuclide may be considered as not present in a mixture if
479		
480		$\underline{a.(a)}$ the ratio of the concentration of that radionuclide in the mixture (CA) to the
481		concentration limit for that radionuclide specified in Appendix A (MPCA) does
482		not exceed $1/10$ (i.e., CA/MPCA \leq than $1/10$), and
483		
484		<u>b.(b)</u> the sum of such ratios for all the radionuclides considered as not present in the
485		mixtures does not exceed $1/4$, (i.e., $(CA/MPCA + CB/MPCB + < \frac{than}{1/4})$.
486		
487		
488		(Source: Amended at 47 Ill. Reg, effective)